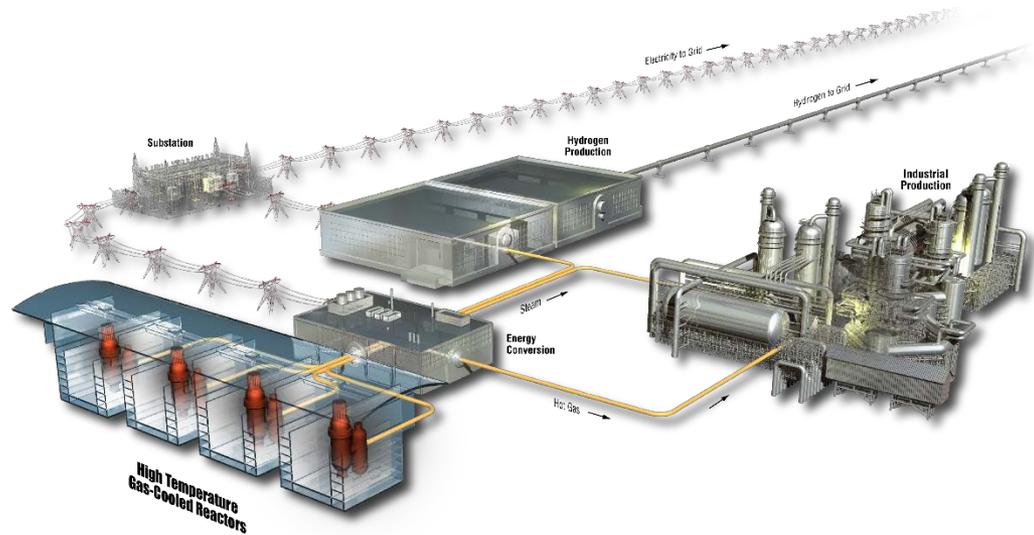


AGR-2 PIE Progress

John Stempien

AGR TRISO Fuels Program Review
Idaho Falls, ID
July 18-19, 2017



www.inl.gov

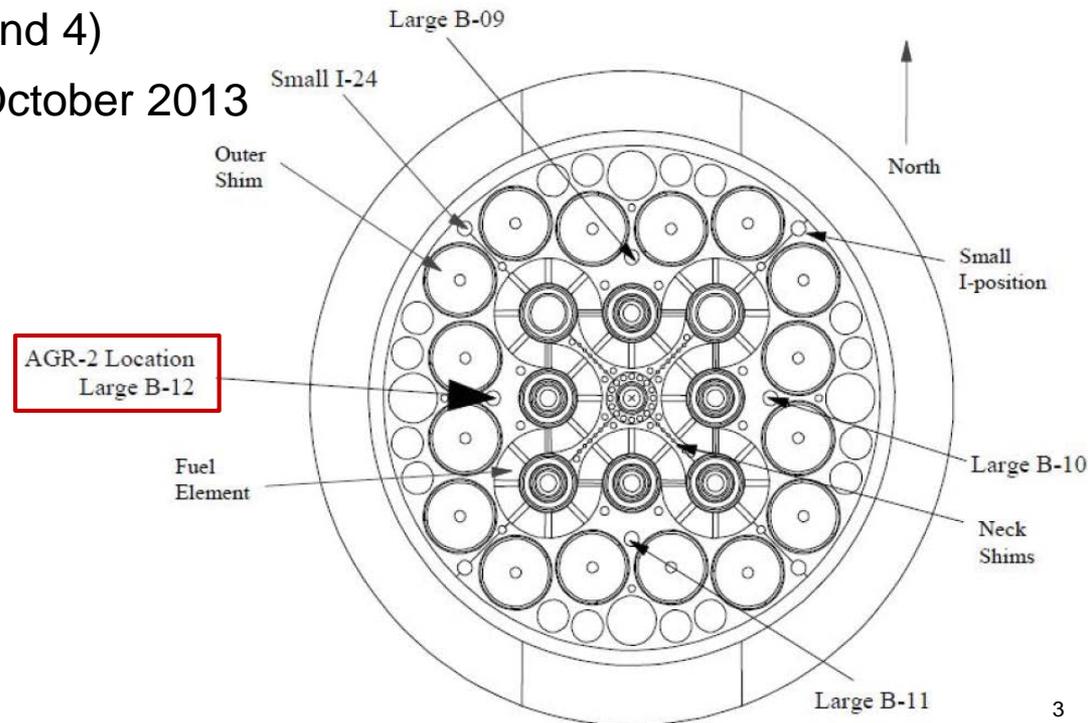


Outline

- AGR-2 irradiation experiment background
 - Fuel samples
 - Irradiation conditions
- Status of analyses on irradiation capsule components
- Fission product mass balance
- Fuel compact non-destructive/destructive analyses and safety testing
- Summary and future work

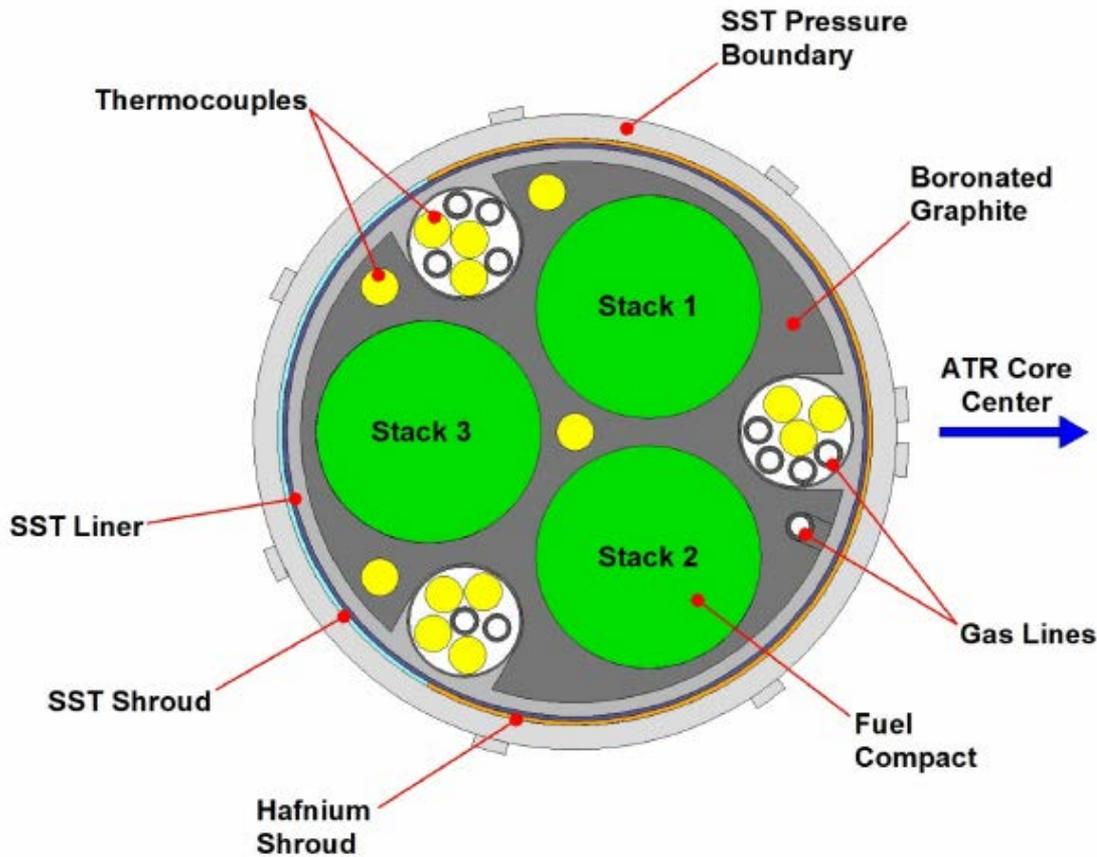
AGR-2 Irradiation

- TRISO fuel particles produced in engineering scale coater (6 in. diameter)
- Total of 6 capsules
- 4 AGR Program Capsules, total of 48 AGR-2 Compacts
 - 36 compacts with AGR-2 UCO fuel kernels (~3176 particles/compact)
 - 12 compacts with UO_2 fuel kernels (~1500 particles/compact) for comparison to UCO and to historic UO_2 data
- 2 CRADA capsules (Capsules 1 and 4)
- Irradiation in ATR: June 2010 to October 2013

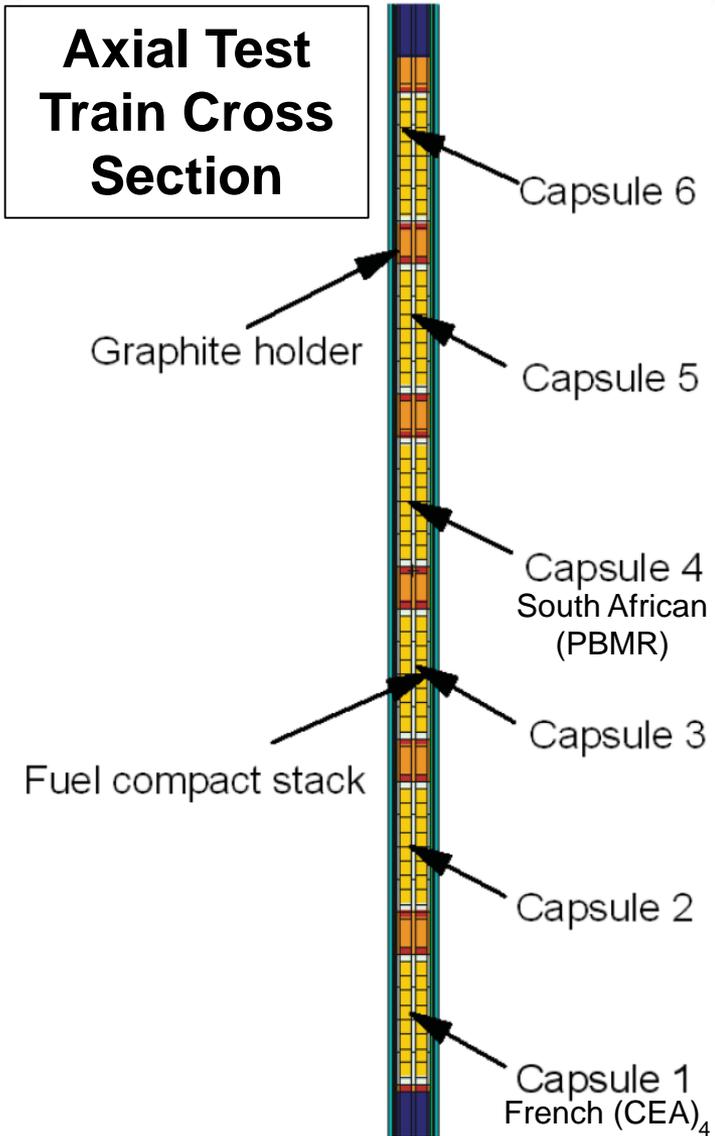


AGR-2 Irradiation Test Train

Radial Capsule Cross Section

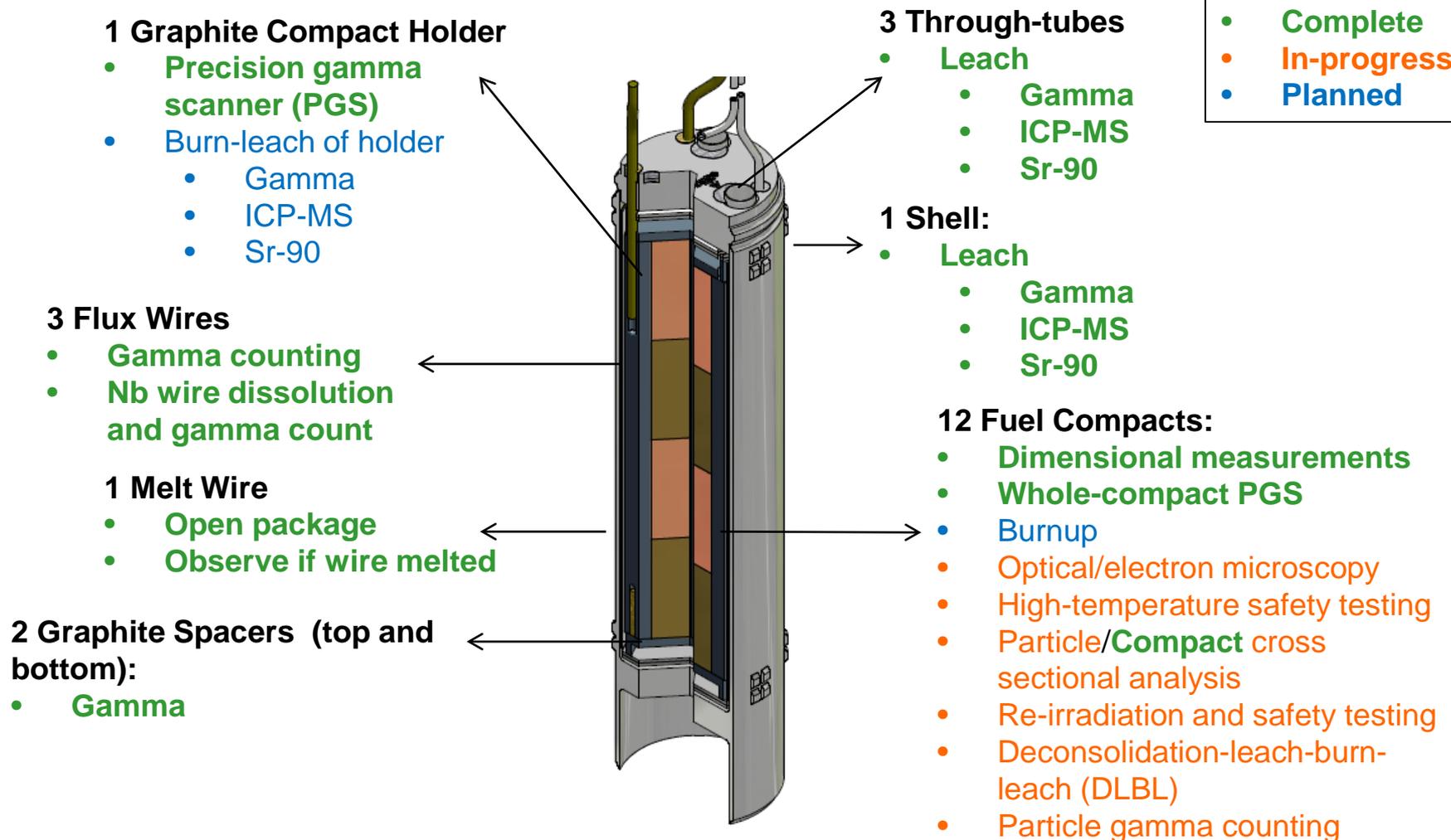


Axial Test Train Cross Section



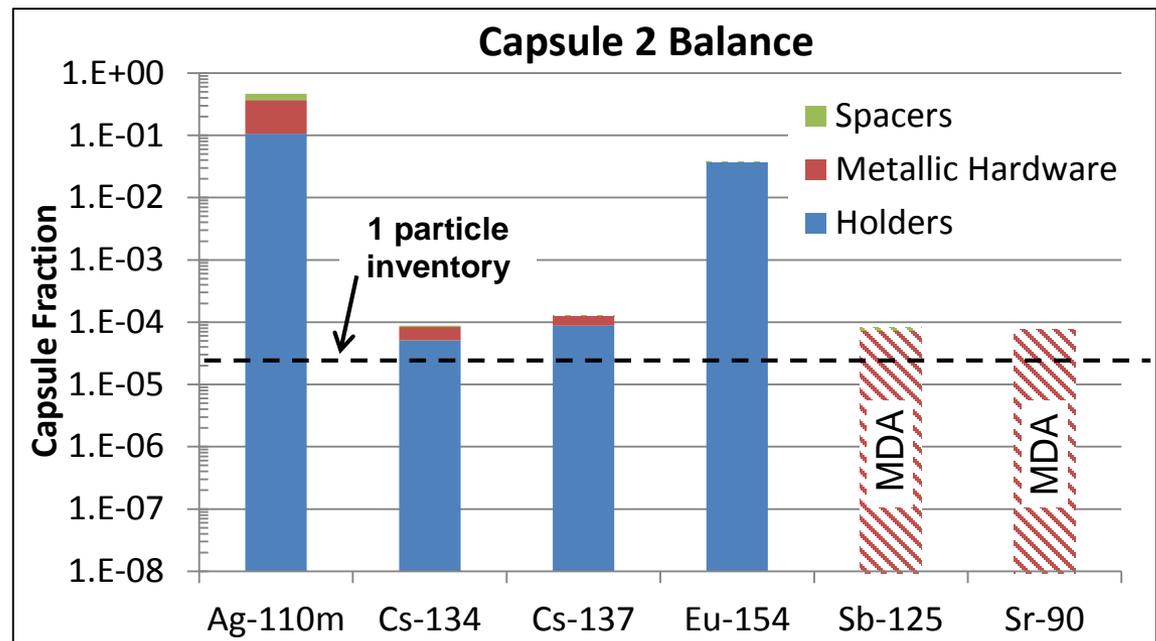
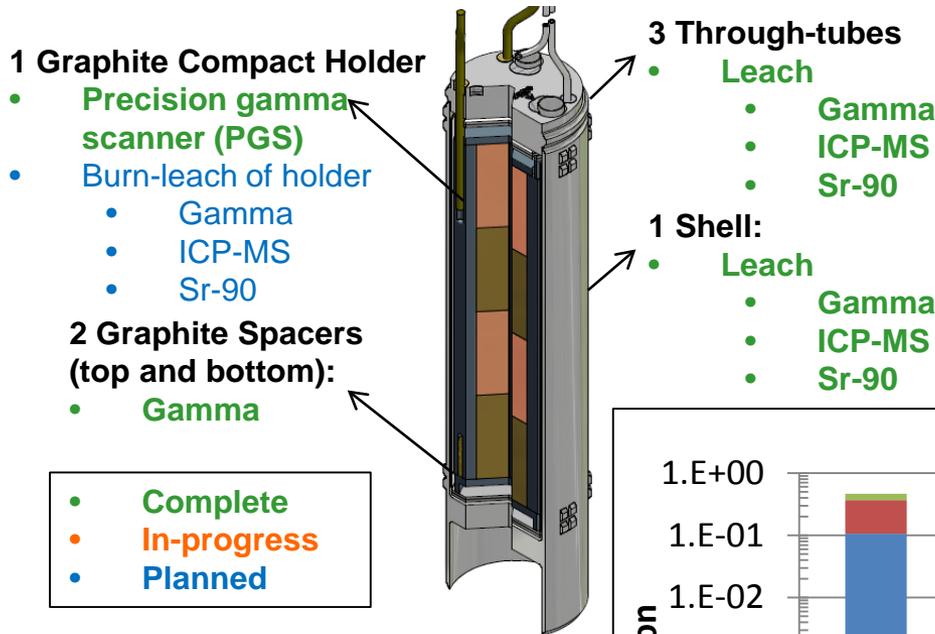
AGR-2 Components and Associated PIE

- Each AGR-2 irradiation capsule has the following components and analyses:



Fission Product Mass Balance Outside of Fuel

- Capsule components from all capsules are analyzed for fission product inventory



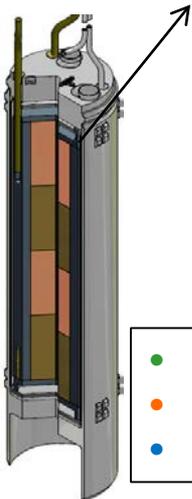
*MDA = minimum detectable activity. No activity was measured. This is the detection threshold.

PGS Helps Locate Potential Fuel Particle Defect

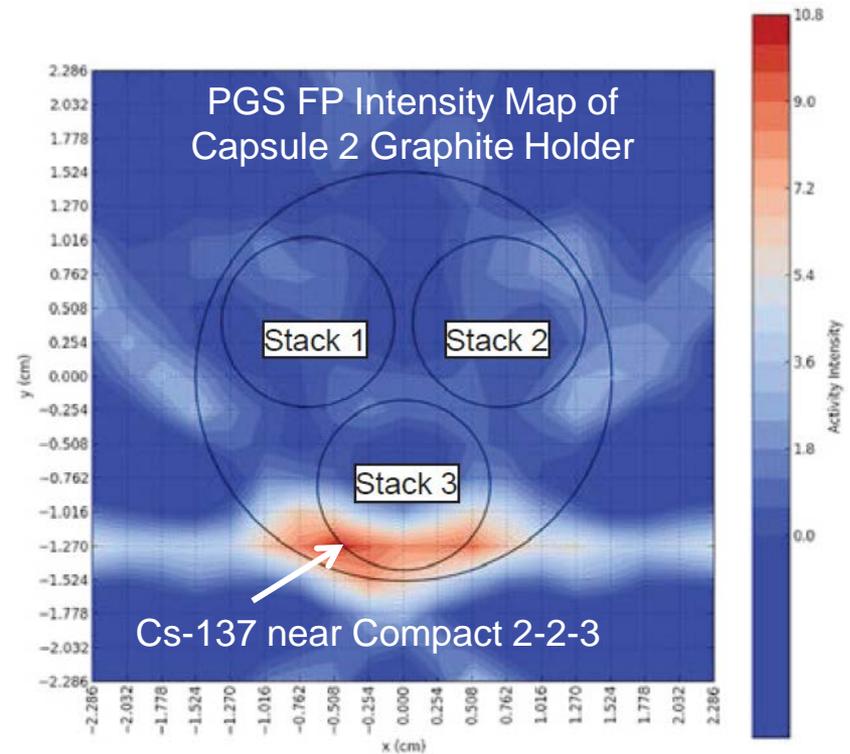
- PGS of graphite holder used to look for spots of elevated fission product activity, fuel compacts near hot spot can be targeted for additional PIE (e.g. Compact 2-2-3)
- Destructive PIE at ORNL found failed SiC layers in a few particles from Compact 2-2-3. Nickel contamination and migration from failed thermocouple near Compact 2-2-3 is believed to be the cause of these SiC failures. Not a fuel quality issue.

1 Graphite Compact Holder

- Precision gamma scanner (PGS)



- Complete
- In-progress
- Planned



Neutron Fluence Wires and Melt Wires

- Fluence wires counted at INL. Some sent to PNNL for comparison.
- Comparison of fluence derived from wires with predicted fluences in-progress
- Melt wires not successfully retrieved from AGR-2 holders; may attempt further inspections of holders for recovery

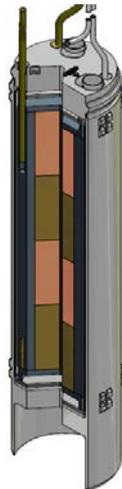
3 Flux Wires ←

- Gamma counting
- Nb wire dissolution and gamma count

1 Melt Wire ←

- Open package
- Observe if wire melted

- Complete
- In-progress
- Planned



U.S. DEPARTMENT OF ENERGY
Prepared for the U.S. Department of Energy
under Contract DE-AC05-76OR01830

PNNL-69575

INTEROFFICE MEMORANDUM Idaho National Laboratory

PNNL Analytical and AGR-2 Monitors SOW-133

Date: November 10, 2016

To: Phil L. Winston
John D. Stempien
Paul W. Humrickhouse
Brian J. Storms

From: David T. Miller *DTM*

Subject: RADIATION MEASUREMENT
RAY ANALYSIS OF THE

Analyst:

Report Preparer: The Advanced Test Reactor (ATR) Compt...
three filter samples each with an absorbed...
counted these wires both on a traditional hi...
of ⁹⁴Nb as well as on two different low-en...
activity by its emitted x-rays. We used exis...
spectrometers, while on the LEPS, we dete...
of ⁹⁴Nb source. Since the source lacks a cali...
uncertainty of 3% based on the precision o...
deviate significantly from Gaussian shape...
direct scaling and chi-square minimization...
which also included an additional constant...
energy gamma rays (i.e. ⁹⁴Nb). To control...
square minimization with all results agree...
These results also agree well with our stan...
The values in Table 1 reflect the time-of-c...
negligible for these long-lived isotopes.

The RML performed quality assurance and...
measurement for the detector systems utili...
counting systems were "in control" for all

Please call if you have any questions or ne...
report (D.T. Miller, 3-4682).

Attachment
cc: L. D. Smith, MS 7111
D. T. Miller Letter File [DTM (FL...]

INL - Materials and Fuels Complex
Analytical Laboratory

Final Report

AL Log #: 99081 SPM # NA
Login Name: AGR-2 FLUX WIRES COC #: NA
Requester: P. DEMKOWICZ, P. HUMRICKHOUSE, D. LAUG, P. WINSTON, J. STEMPIEN Charge #: 102421420

Facility: HFEF, Bldg. 785 Date Received: 11-Nov-15 1:03:04 P
Approved by: *[Signature]* Date: *12 JAN 16*
Total Samples in Report: 18

Sample ID: BOTFW-13, AGR 2 FLUX WIR Sampling Date: 11/5/2015
Where Taken: HFEF
Description: SOLID VANADIUM CAPSULE WITH COBALT, IRON OR NIOBIUM WIRE (2MG EACH) INSIDE IT

Analytical Method	Analyte	Result	Units	Error @ 2 Sigma
Gamma Specs				
Ce-60		1.89e-1	uCi/sample	±3%
Fe-59		<9e-3	uCi/sample	N/A
Mn-54		<9e-3	uCi/sample	N/A
Nb-94		<9e-3	uCi/sample	N/A
Nb-95		<9e-3	uCi/sample	N/A
Sc-46		<9e-3	uCi/sample	N/A
Ta-182		<9e-3	uCi/sample	N/A

Comments: Counted 12-1-15 4:54PM

Sample ID: BOTFW-14, AGR 2 FLUX WIRE Sampling Date: 11/5/2015
Where Taken: HFEF
Description: SOLID VANADIUM CAPSULE WITH COBALT, IRON OR NIOBIUM WIRE (2MG EACH) INSIDE IT

Analytical Method	Analyte	Result	Units	Error @ 2 Sigma
Gamma Specs				
Ce-60		2.37e+0	uCi/sample	±3%
Fe-59		<9e-3	uCi/sample	N/A
Mn-54		<9e-3	uCi/sample	N/A
Nb-94		<7e-3	uCi/sample	N/A
Nb-95		<9e-3	uCi/sample	N/A
Sc-46		<9e-3	uCi/sample	N/A

Fuel Compact Analyses

- Majority of PIE focuses on fuel compacts
- Large test/exam matrix
 - Each capsule had different burnup/irradiation temperature ranges
 - One capsule had UO₂ fuel for comparing to AGR UCO fuel performance

12 Fuel Compacts per Capsule:

- Dimensional measurements
- Whole-compact PGS
- Burnup
- High-temperature safety testing
- Optical/electron microscopy
- Particle/**Compact** cross sectional analysis
- Re-irradiation
- Deconsolidation-leach-burn-leach (DLBL)
- Particle gamma counting

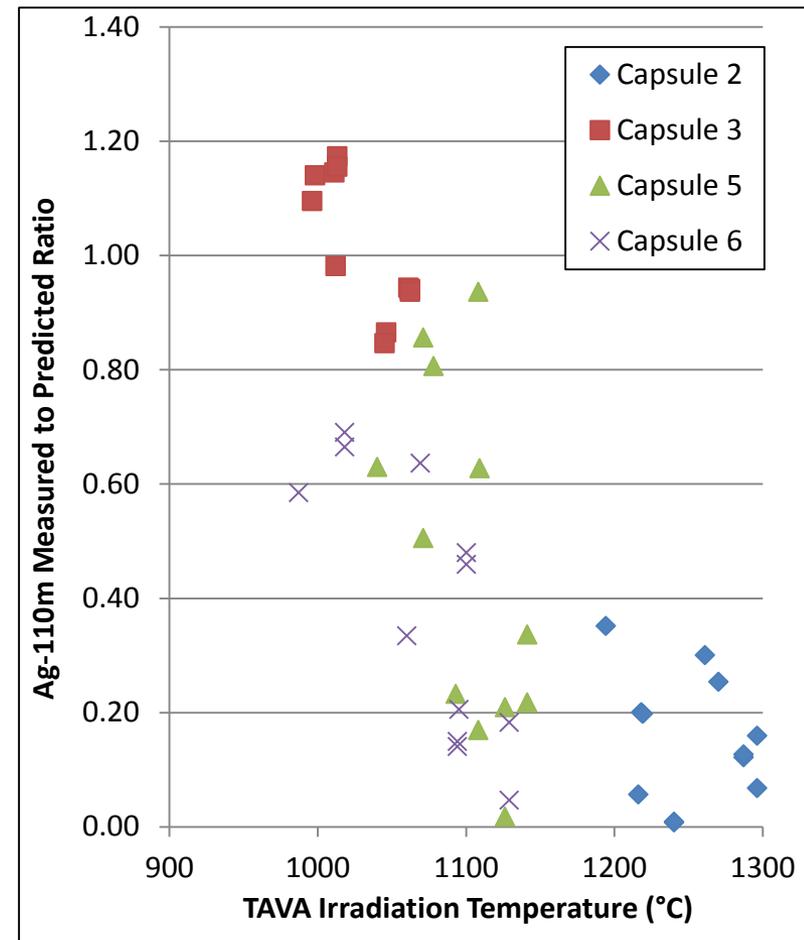
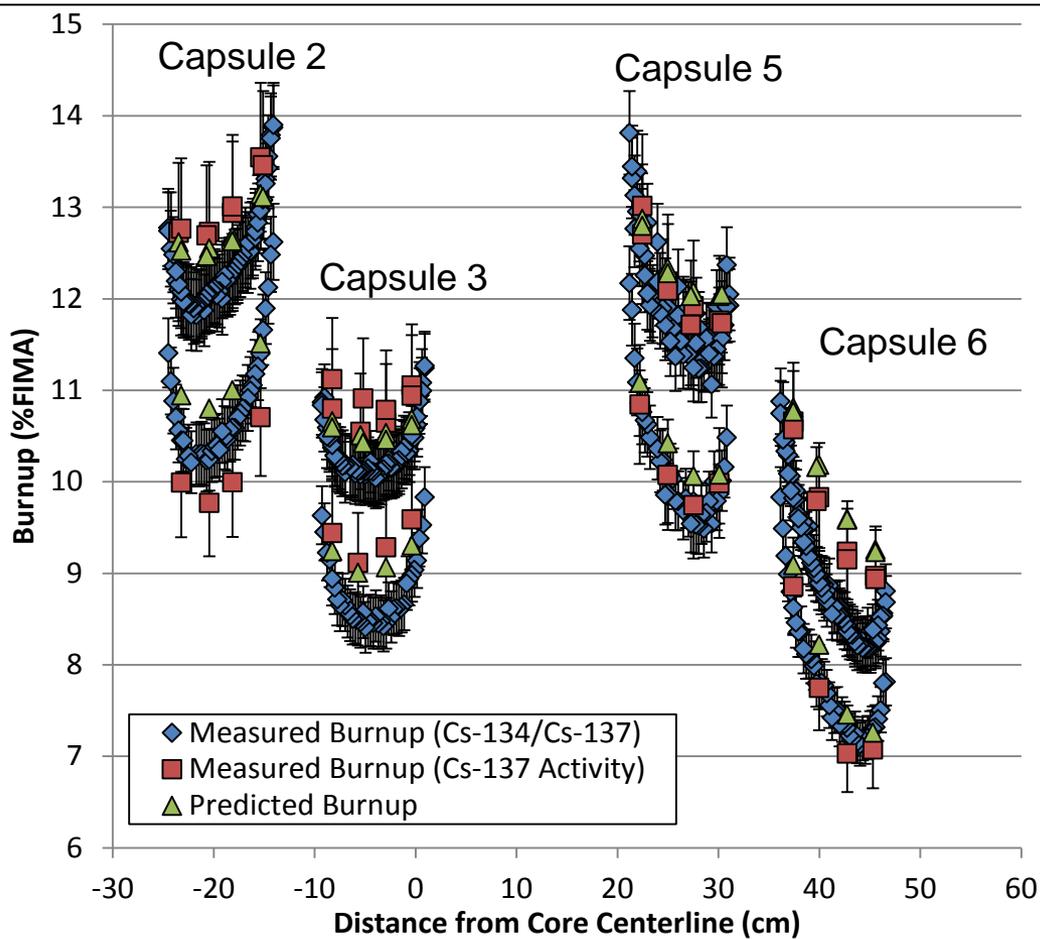
- | |
|--|
| <ul style="list-style-type: none"> • Complete • In-progress • Planned |
|--|

Fuel Compact PGS

- Estimate burnup
- Compare measured inventories to predicted (e.g. Ag-110m)

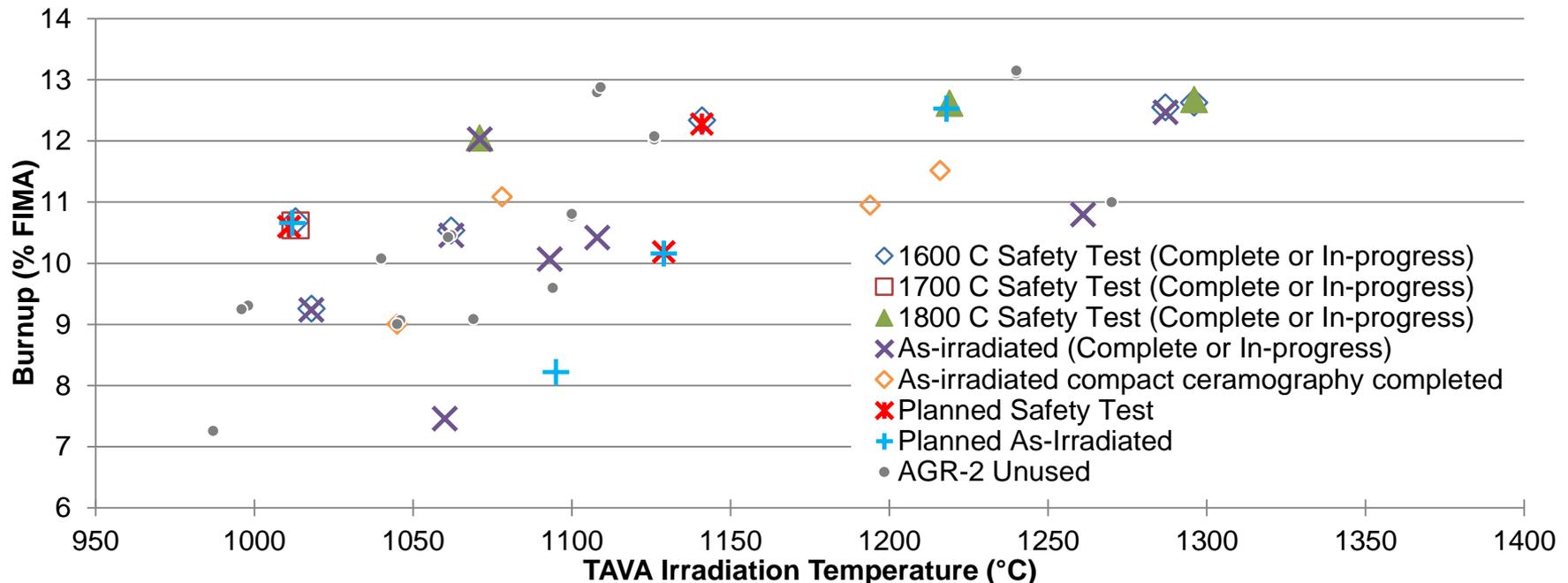
- Complete
- In-progress
- Planned

- 12 Fuel Compacts per Capsule:
- Whole-compact PGS
 - Burnup by chemistry methods



Compact Safety Testing, Destructive, and Microscopy Cover Range of Irradiation Conditions

- Completed 10 safety tests (3 in FY17 for **L2 Milestone** to “complete safety testing of 2 compacts”)
- Completed 5 as-irradiated analyses (including 4 compact ceramography), 7 more in-progress
- The following presentations will elaborate on these activities in more detail:
 - “Re-irradiation Testing”, Paul Demkowicz
 - “AGR-2 Fuel Compact Destructive Exams”, John Hunn
 - “AGR-2 Safety Testing”, Bob Morris
 - “AGR Advanced Microscopy”, Isabella van Rooyen
 - “Long duration heating tests for silver release at intermediate temperatures,” Tyler Gerczak

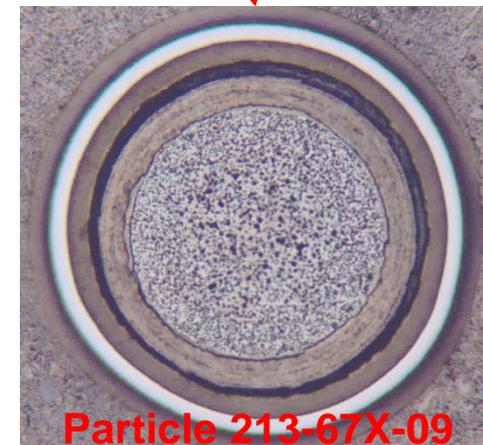
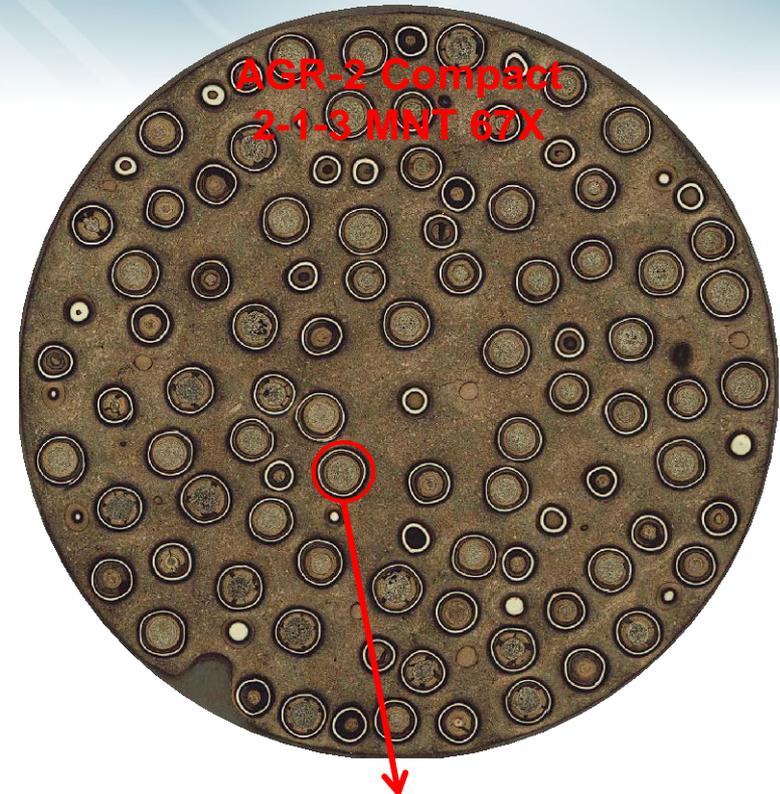


AGR-2 Compact Ceramography

- Examined 538 UCO particles
 - Compacts 2-1-3, 2-4-3, and 5-1-3
- Examined 102 UO₂ particles
 - Compact 3-2-3
- Compared to AGR-1:

	AGR-1	AGR-2
Partial IPyC Tears (%)	2.3	3.5
Through-layer IPyC Fracture (%)	3.5	None
Complete and Partial Buffer-IPyC Separation (%)	94	100
Buffer Fracture (%)	23	20

- Evidence that longer fluidization time between buffer layer deposition and IPyC layer deposition during fabrication promotes buffer-IPyC separation
- Easy and complete buffer-IPyC separation desirable
- Partial separation concentrates stresses at IPyC layer
- Also observed decreasing buffer fracture frequency as irradiation temperature increased above 1080°C to 1220°C



Summary

- PGS has been used to identify compacts that may have degraded SiC layers
 - 5 Compacts (2-2-3, 5-2-3, 5-3-3, 6-3-3, and 6-2-3)
 - 2-2-3: possibly 1 as-fabricated TRISO defect and 3 -5 SiC failures due to Ni contamination from a failed thermocouple
 - 5-2-3: possibly 2 or 3 particles with failed coatings
 - Others still in analysis
- Flux wires retrieved and counted. Comparison to model predictions in-progress
- Completed Compact Analyses/Tests:
 - PGS of all compacts
 - 10 safety tests
 - Cross-section ceramography showing desirable enhanced buffer-IPyC separation
 - 10 as-irradiated compact analyses (including compact ceramography)

Future Work

- Fission product mass balance: **remaining planned work** is burn-leach of graphite holders for Sr-90
- Destructive compact analyses:
 - Safety testing: **planning** to complete **at least 3 more** safety tests before end of FY18
 - As-irradiated: **planning** to complete **at least 4 more** before end of FY18
- Particle analyses:
 - **Planning** to do cross sectional analysis of 100 particles from each of 4 compacts for kernel swelling and buffer shrinkage (beginning in FY17)
 - **Continue** advanced microscopy of kernel and layer samples from particles from 2 compacts
 - **Begin** long-term (~1000 hrs +) heating test of loose particles for long-term fission product release measurements

Questions and Discussion

John Stempien

Idaho National Laboratory

john.stempien@inl.gov

(208) 526-8410



AGR-2 Components and Associated PIE

- Each AGR-2 irradiation capsule has the following components and analyses:

1 Graphite Compact Holder

- Precision gamma scanning (PGS)
- Burn-leach of holder
 - Gamma
 - ICP-MS
 - Sr-90

3 Flux Wires

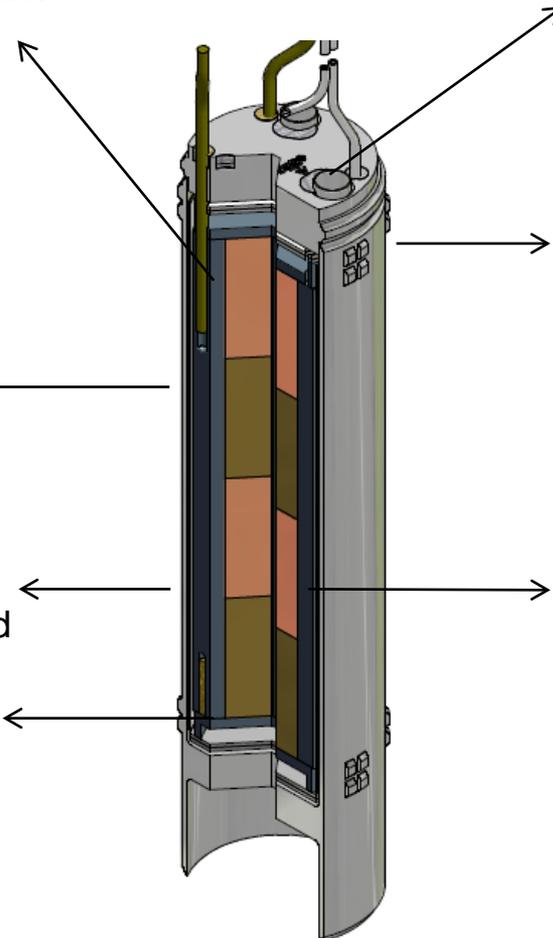
- Gamma counting
- Nb wire dissolution and gamma

1 Melt Wire

- Open package
- Observe if wire melted

2 Graphite Spacers (top and bottom):

- Gamma



3 Through-tubes

- Leach
 - Gamma
 - ICP-MS
 - Sr-90

1 Shell:

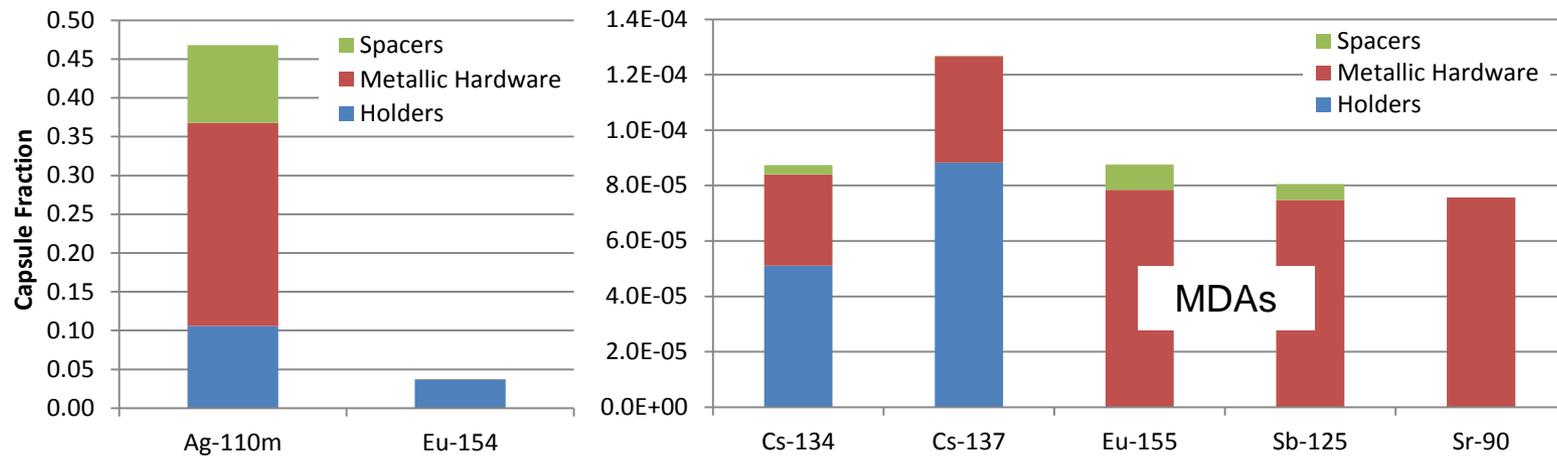
- Leach
 - Gamma
 - ICP-MS
 - Sr-90

12 Fuel Compacts:

- Dimensional measurements
- Whole-compact PGS
- Burnup
- Optical/electron microscopy
- High-temperature safety testing
- Particle/Compact cross sectional analysis
- Re-irradiation
- Deconsolidation-leach-burn-leach (DLBL)
- Particle gamma counting

Example of Recent AGR-2 FP Analysis

- Capsule 2: UCO, TAVA Temp 1194°C



- Capsule 3: UO₂, TAVA Temp 1045°C

